

Dosimetry of ionizing radiations-1: dose concepts, radiation therapy

03-18-2021

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Radiation = spreading of energy

energy > ionization energy:

ionizing radiation

Ionizing radiation

Classification according to the primary effect



Direct ionization

Incoming particles are charged and ionize until losing their energy.
(α - and β -particles, protons, ions)

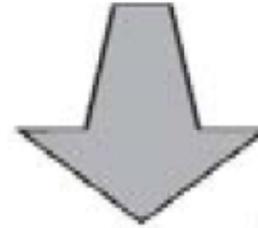
Indirect ionization

Primary electrons ejected by the incoming radiation and secondary electrons ionize further.
(γ and X ray photons, neutrons)

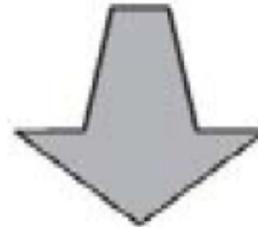
Ionizing radiation



Discovery
(X-ray, radioactivity etc.)



Application
(enjoy benefits)



Dosimetry
(optimization of benefits,
estimation of risk and hazard)



Tasks for dosimetry

Estimation of health risk for prevention.

Estimation of biological damages.

Design of therapeutic procedures.

*Definition of
quantities*



*Design of
measuring
techniques*



*Estimation of
consequences*

1. Dose values should be

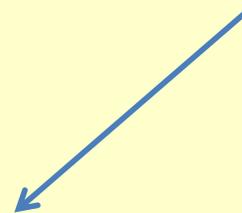
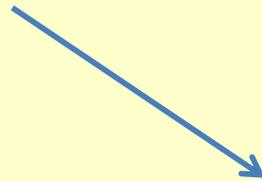
- proportional to the damages and expected risk
- additive
- independent of other factors

Dose concepts

Physical dose concepts:
Absorbed dose,
Exposure

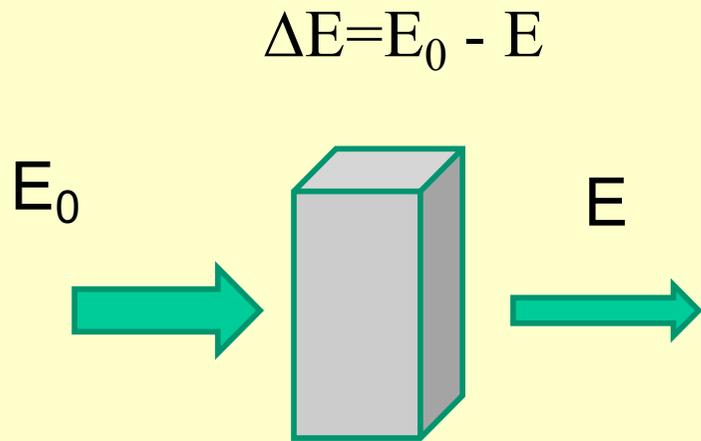
Biological dose concepts:
Equivalent dose,
Effective dose

Derived dose concepts:
Collective dose,
Dose rate



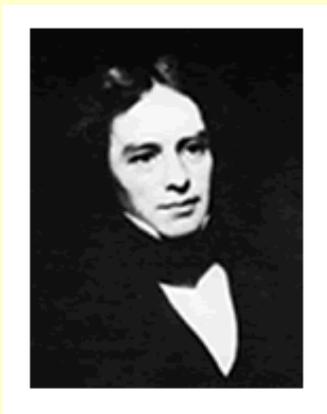
1. Absorbed dose

measures the absorbed energy in a unit mass



$$D = \frac{\Delta E}{\Delta m} [J / kg]$$

Validity: for any kind of material and any type of radiation without restriction



Louis Harold Gray
(1905-1965).

Unit:

$$[J / kg] \equiv Gy$$

How to be measured?!

^{131}I of 0.2 GBq activity is accumulated in 80 g thyroid glands. The effective half-life is 7.5 days . Average β -particle energy is 0.18 MeV. Assume that the particles are fully absorbed in the thyroid glands . What is the absorbed dose in the given tissue?

$$\Lambda = \frac{\ln 2}{T} N$$

$$N = \frac{0,2 * 10^9 [\text{Bq}] * 6,48 * 10^5 [\text{s}]}{0,693} = 1,87 * 10^{14}$$

$$E_{\text{sum}} = N * E$$

$$E = 0,18 * 10^6 [\text{eV}] = 2,88 * 10^{-14} [\text{J}]$$

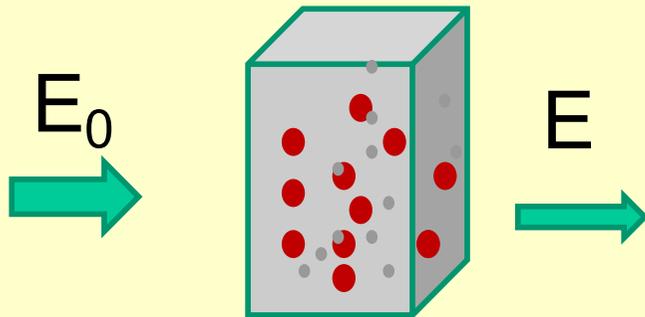
$$E_{\text{össz}} = 1,87 * 10^{14} * 2,88 * 10^{-14} = 5,38 [\text{J}]$$

$$D = \frac{E_{\text{sum}}}{m}$$

$$D = \frac{5,38}{0,08} = 67,28 \left[\frac{\text{J}}{\text{kg}} \right]$$

2. Exposure

measures the amount of positive or negative charges generated by the radiation in a unit mass.



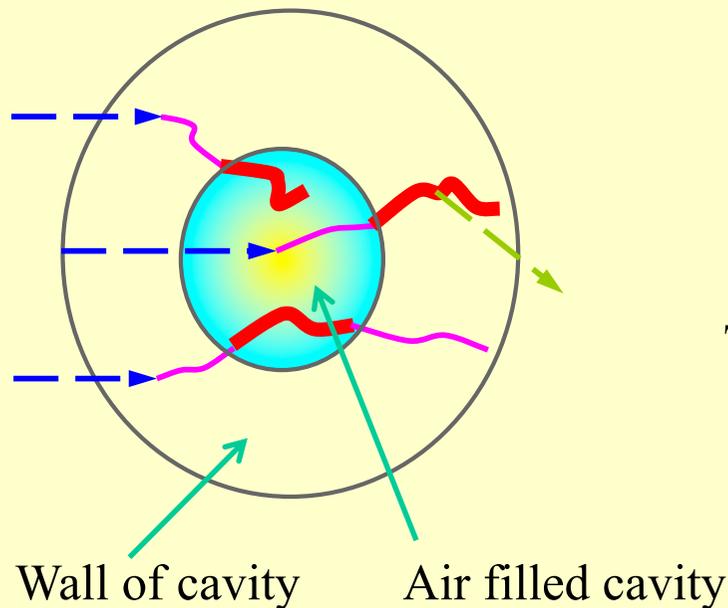
$$X = \frac{\Delta Q}{\Delta m} [C / kg]$$

Validity: in the air, only γ and X-rays, measured in electron equilibrium*

$$X = \frac{\Delta Q}{\Delta m} [C / kg]$$

ΔQ – secondary electrons!!

Electron-equilibrium: net number of the secondary electrons living and entering volume of the cavity are equal.



To be considered:

- composition of surrounding material
(chamber wall) –

air-equivalent wall!

- thickness of the wall

- Photon energy: $E < 0.6 \text{ MeV}$

Calculation of the absorbed dose from the exposure

$$X = \frac{\Delta Q}{\Delta m} [C / kg]$$

$$D_{\text{air}} = f_0 X$$

$$D = \frac{\Delta E}{\Delta m} [J / kg]$$

~ 34 J/C

Average ionization energy in air

~ 34 eV.

Absorbed dose in tissue

$$\frac{\Delta E}{\Delta m} \approx \mu_m \cdot J$$

$$D_{\text{air}} = \frac{\Delta E}{\Delta m} [\text{J / kg}]$$

$$\frac{D_{\text{air}}}{D_{\text{tissue}}} = \frac{\mu_{\text{m,air}}}{\mu_{\text{m,tissue}}}$$

Photon energy (MeV)	$\mu_{\text{m,air.}}/\mu_{\text{m,tissue}}$ (soft tissues)	$\mu_{\text{m,air.}}/\mu_{\text{m,tissue}}$ (bones)
0,1	1,07	3,54
0,2	1,08	2,04
0,4	1,10	1,24

Biological dose concepts

Equivalent dose

Effective dose

The absorbed energy (absorbed dose) is not sufficient to measure the possible biological consequences.

The biological consequences are influenced by :

the type of radiation.



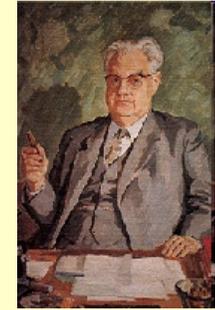
radiation weighting factor

the sensitivity and biological function of target



tissue weighting factor

Rolf Sievert
1896-1966



Equivalent dose (H)

„Efficiency” of various forms of radiation is not uniform.

$$H_T = w_R D_T$$

Radiation weighting factor – estimation of the relative risk of the given radiation

Absorbed dose
in tissue

Unit of H : *Sievert (Sv)*

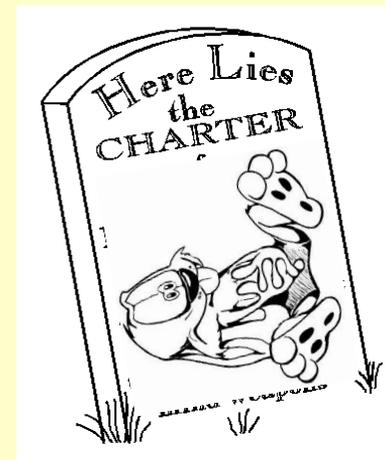
radiation	w_R
photon	1
electron	1
neutron	5-20
proton	5
α -particle	20

Why are the fates of the rabbits different?

2 Gy absorbed dose – *X-ray*

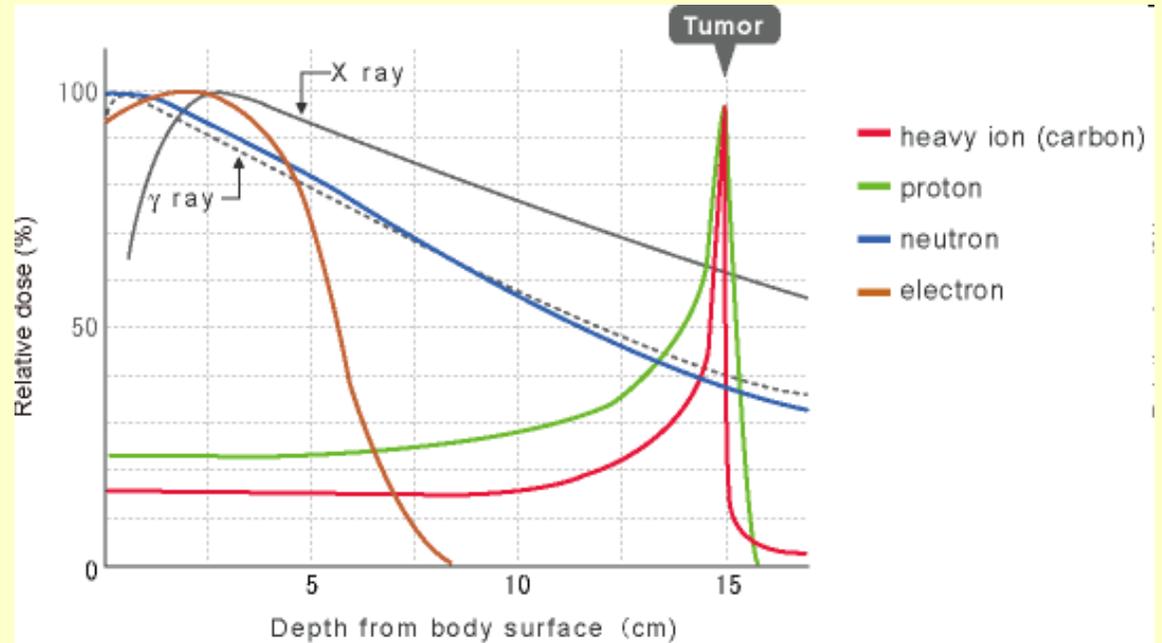
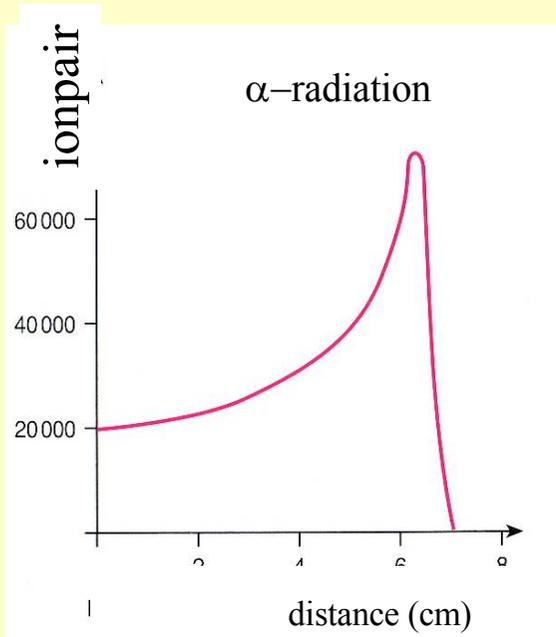


2 Gy absorbed dose – *α -particles*



Equivalent dose (H)

„Efficiency” of various forms of radiation is not uniform.

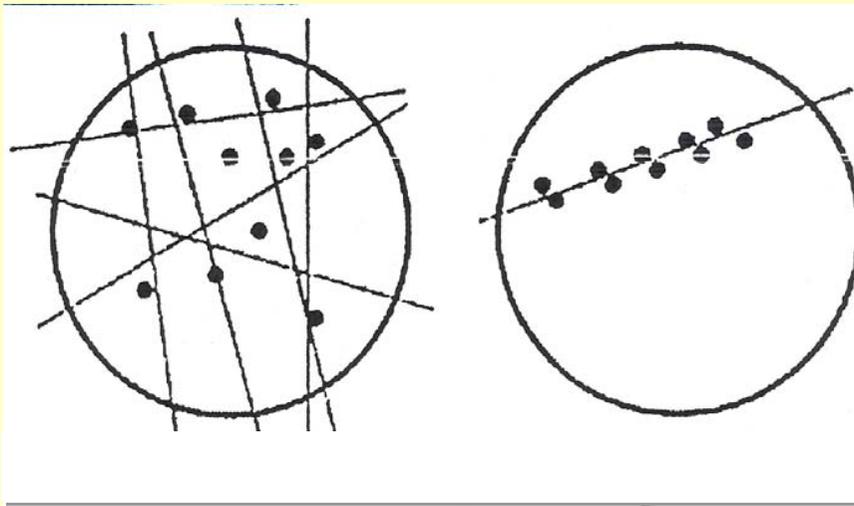


LET (Linear Energy Transfer: the energy transferred to the material surrounding the particle track, by means of secondary electrons. $(nE_{ionpair}/l)$

Equivalent dose (H)

„Efficiency” of various forms of radiation is not uniform.

$$H_T = w_R D_T$$

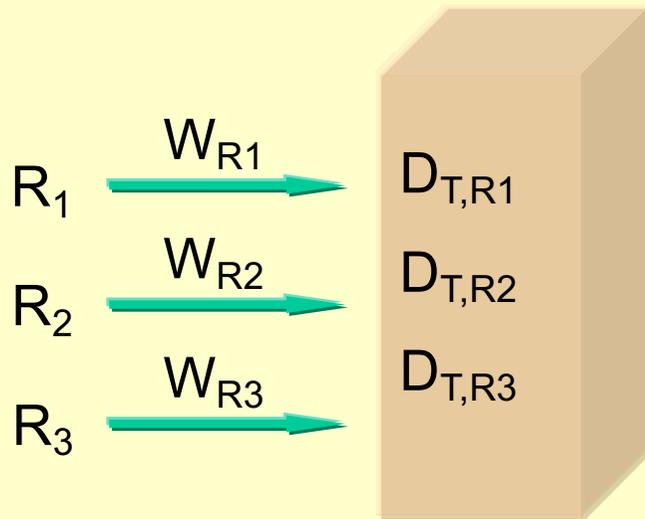


Small LET
e.g. γ , X-ray

High LET
e.g. α , proton

radiation	w_R
photon	1
electron	1
neutron	5-20
proton	5
α -particle	20

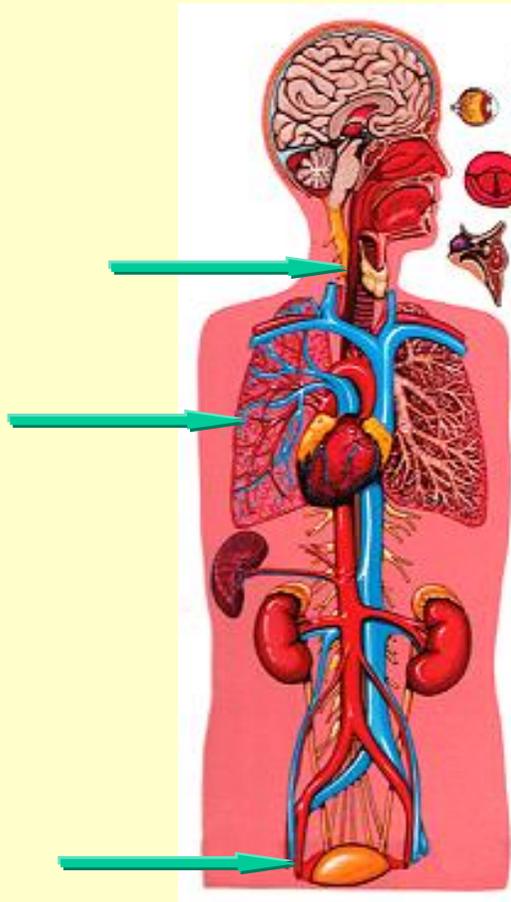
If someone is exposed to more than one type of radiation:



$$H_T = \sum_R w_R D_{T,R}$$

Effective dose (E)

Various sensitivity of tissues has to be considered



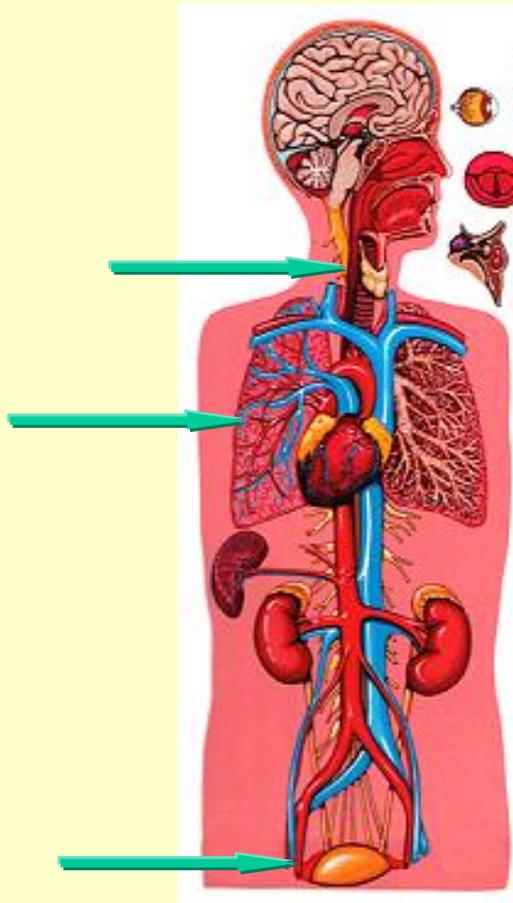
$$E = \sum_T w_T H_T$$

Tissue weighting factor
– estimation of the relative sensitivity of tissue

Equivalent dose in the given tissue

Unit of E : *Sievert (Sv)*

$$E = \sum_T w_T H_T$$

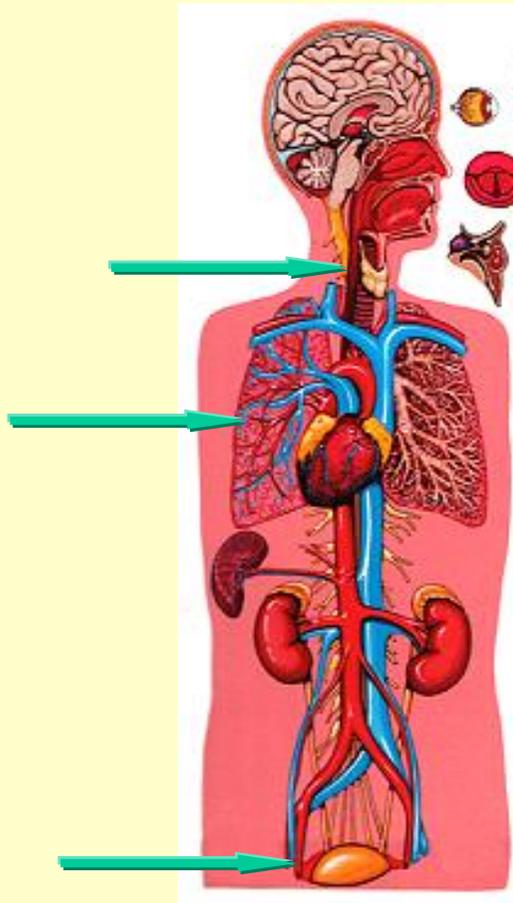


tissue	w_T	tissue	w_T
gonads	0,2	breast	0,05
bone marrow	0,12	liver	0,05
colon	0,12	oesophagus	0,05
lung	0,12	thyroid gland	0,05
stomach	0,12	skin	0,01
bladder	0,05	bone surface	0,01

$$\sum_T w_T = 1$$

$$E = \sum_T w_T H_T$$

$$\sum_T w_T = 1$$



Organ or tissue	W_T ICRP 30 (1979) ^a	W_T ICRP 60 (1991)	W_T ICRP 103 (2007)
Gonads	0.25	0.20	0.08
Red bone marrow	0.12	0.12	0.12
Large intestine		0.12	0.12
Lung	0.12	0.12	0.12
Stomach		0.12	0.12
Bladder		0.05	0.04
Breast	0.15	0.05	0.12
Liver		0.05	0.04
Oesophagus		0.05	0.04
Thyroid	0.03	0.05	0.04
Skin		0.01	0.01
Bone surface	0.03	0.01	0.01
Rest ^b	0.30	0.05	0.12
Brain			0.01
Total	1.00	1.00	1.00

^a ICRP 30 W_T are used to calculate EDE, whereas ICPR 60 W_T and ICRP 103 W_T give E values.

^b 'Rest' includes adrenals, small intestine, kidney, muscle, brain (except ICRP 103 W_T), pancreas, spleen, thymus and uterus.

Dose rate

Received dose over time.

Unit: varies with the type of radiation and the time period (pl. Gy/month, mSv/year etc.)

Collective dose

Sum of the doses received by a given number of people (N_i) in the course of a given time interval.

Collective dose

Sum of the doses received by a given number of people (N_i) in the course of a given time interval.

$$S = \sum_i N_i E_i$$

E_i effective dose in each person

Consider an α -emitting isotope of 5 MBq activity. The energy of the emitted α particles is 6.2 MeV. The total emitted energy is absorbed in 0.1 kg water. Calculate the the absorbed dose in the water after half an hour irradiation. (There is no significant change in the activity of the radioactive sample during the time of the experiment.)

$$\Lambda = 5 \text{ MBq, azaz } 5 * 10^6 \text{ decay/s}$$

$$N = 5 * 10^6 * 1800 = 9 * 10^9 \text{ decay in half an hour}$$

$$E_{\text{abs}} = E_{\alpha} * N \quad E_{\text{abs}} = 6.2 * 10^6 \text{ eV} * 9 * 10^9$$

$$E_{\text{abs}} = 5,58 * 10^{16} \text{ eV} = 8,92 * 10^{-3} \text{ J}$$

$$D = \frac{E_{\text{abs}}}{m} = \frac{8.92 * 10^{-3}}{0.1} = 8.92 * 10^{-2} [\text{Gy}]$$

Consider an α -emitting isotope of 5 MBq activity. The energy of the emitted α particles is 6.2 MeV. The total emitted energy is absorbed in 0.1 kg water. Calculate the temperature change of the water after half an hour irradiation. (There is no significant change in the activity of the radioactive sample during the time of the experiment.)

$$\Lambda = 5 \text{ MBq, azaz } 5 \cdot 10^6 \text{ decay/s}$$

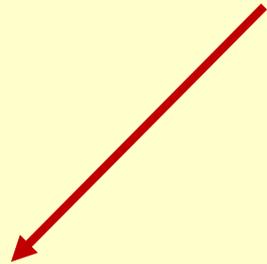
$$N = 5 \cdot 10^6 \cdot 1800 = 9 \cdot 10^9 \text{ decay in half an hour}$$

$$E_{\text{abs}} = E_{\alpha} \cdot N \quad E_{\text{abs}} = 6.2 \cdot 10^6 \text{ eV} \cdot 9 \cdot 10^9$$

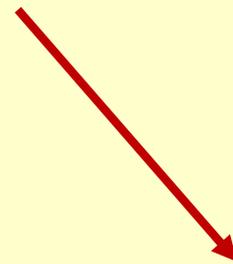
$$E_{\text{abs}} = 5,58 \cdot 10^{16} \text{ eV} = 8,92 \cdot 10^{-3} \text{ J}$$

$$E_{\text{abs}} = c \cdot m \cdot \Delta T \quad \Delta T = \frac{8.92 \cdot 10^{-3}}{4.18 \cdot 10^{3 \cdot 0.1}} = 2.1 \cdot 10^{-5}$$

Types of damages

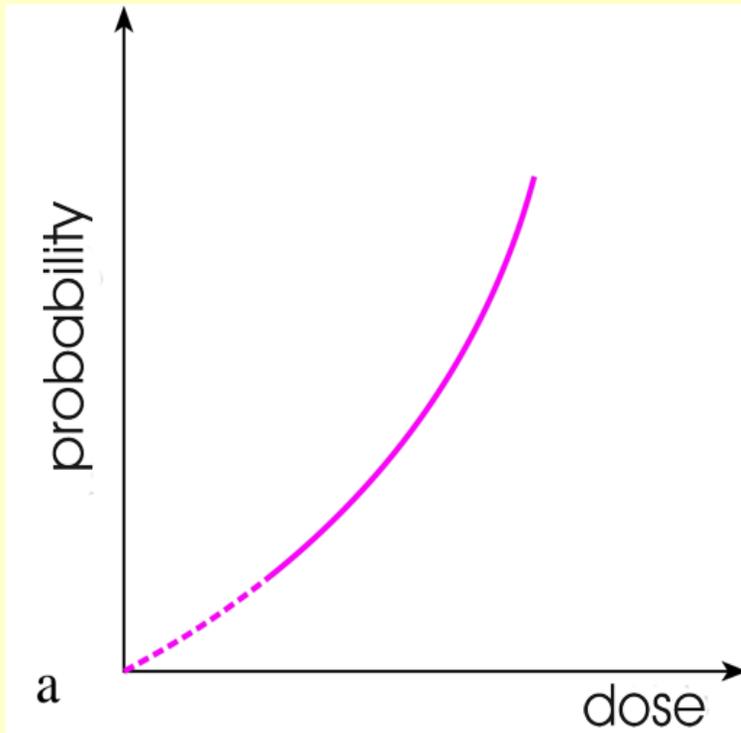


Stochastic damages



Deterministic damages

Stochastic damages



NO threshold!

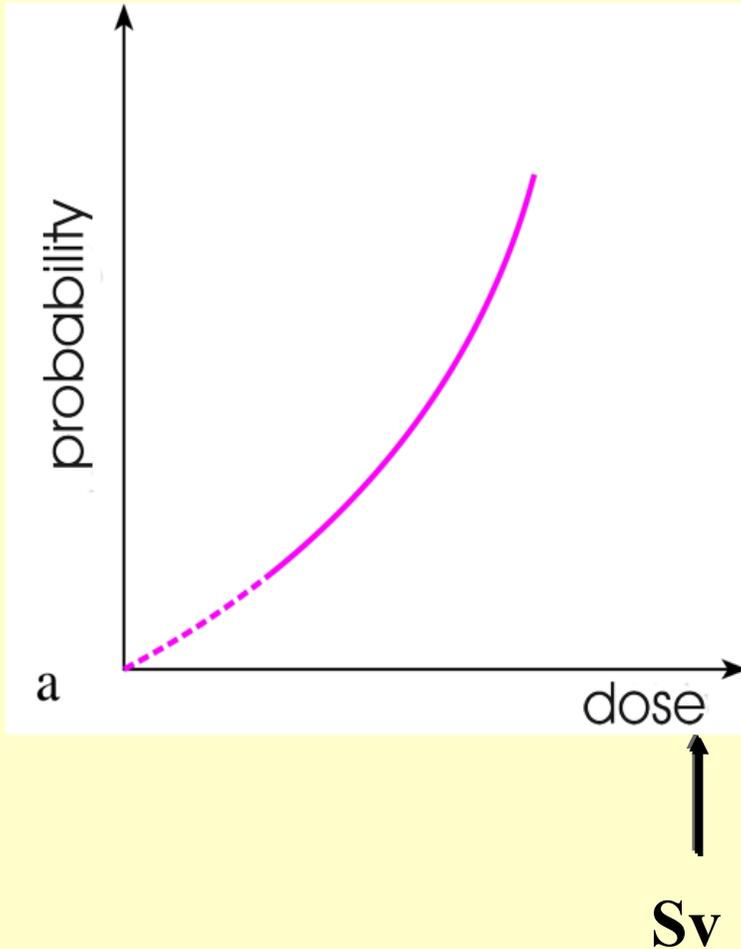
The probability of stochastic damage depends on the dose.

Severity (e.g. cancer) independent of the dose.

Delayed biological effects.

e.g. tumours, hereditary diseases

Stochastic damages



H_T (equivalent dose) and E (effective dose) provide a basis for *estimating the probability of stochastic effects* for doses below the threshold of deterministic effects.

Dose range is under the threshold of deterministic damages.

Stochastic damages

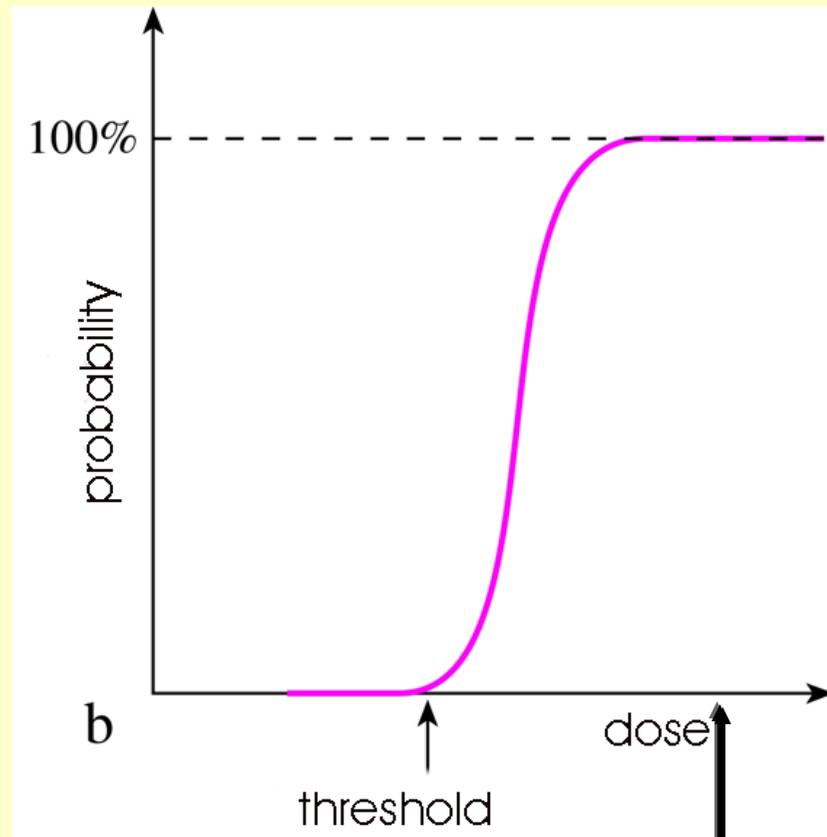
Irradiated cell is modified rather than killed

Severity is not affected by the dose

With increasing dose only the probability* increases

*1 gamma photon: the probability to cause cancer is 3×10^{-16} (1: 3,000 billion, but this is a Russian roulette!)

Deterministic damages



Gy

Deterministic damages

A threshold dose exists.

Above threshold severity depends on the dose.

Appear soon after exposition.

Must not be induced during diagnostic procedures.

e.g. erythema, epilation, cataract

*1% lethal 60 days after exposition

Dose (Gy) (whole body)	Biological effect
< 0,15-0,2	No observable effect
0,5	Slight blood changes – limit of detection by hematological methods.
0,8	Critical dose – threshold of acute radiation syndrome
2,0	Minimal lethal dose (LD1/60)*
4,0	Half lethal dose (LD50/60)
7,0	Minimal absolute lethal dose (LD99/60)

Chest X-ray: cc 160 μ Gy in the skin

Radiotherapy

Which radiation is the best?

What is the optimal dose of radiation?

What is the best technique for generating radiation?

Irradiation selectivity – protection of healthy structures?



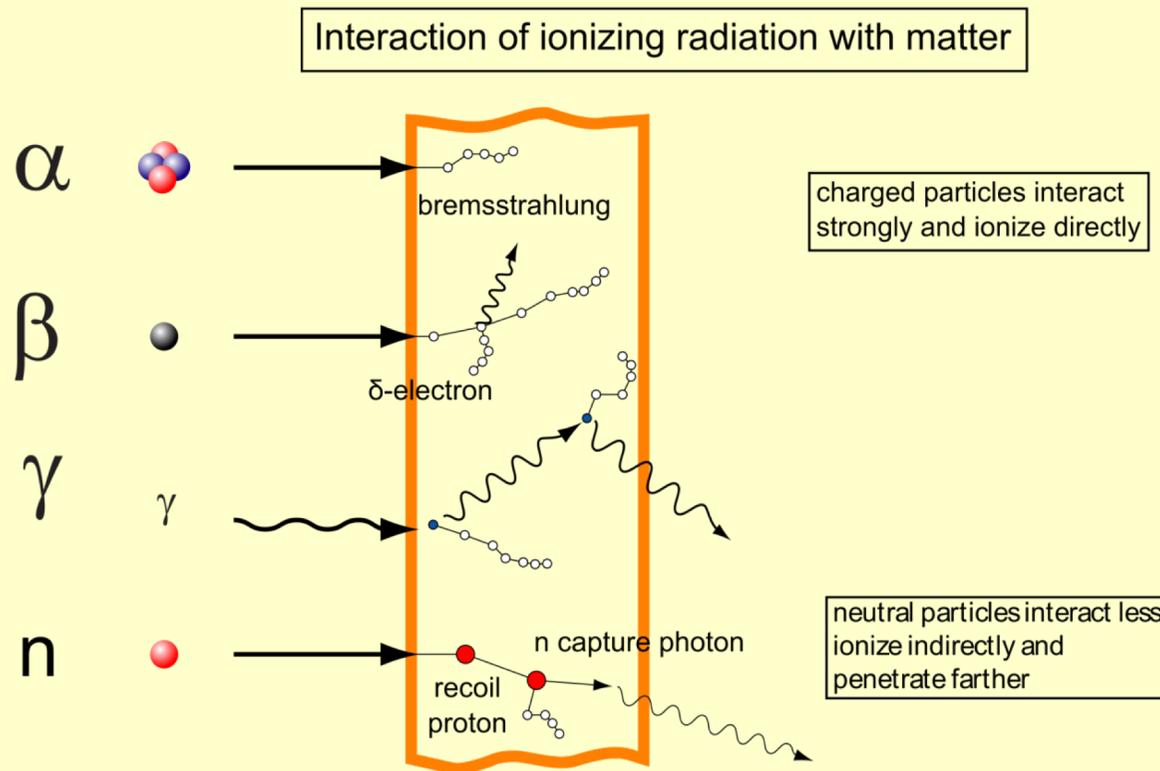
Radiation therapy is a clinical modality dealing with the use of ionizing radiations in the treatment of patients with malignant neoplasias (and occasionally benign diseases).

The **aim of radiation therapy** is to deliver a **curative** dose of irradiation to a defined tumor volume with as minimal damage as possible to surrounding healthy tissue.

Consequences of the absorption of ionizing radiation.

1. Physical events

Direct or indirect ionization



The amount of secondary ionization depends on the material; it can be up to 10 times the amount of primary ionization.

The gamma photon emitted by the nucleus of the cesium isotope with 137 mass number is absorbed with photoeffect. The absorbing medium is air, assume the work function to be 34 eV. What will be the kinetic energy of the photoelectron in eV?

$$E_{^{137}\text{Cs}} = 0,661\text{MeV}$$

$$hf = A + \frac{1}{2}mv^2$$

$$\frac{1}{2}mv^2 \approx 661\,000\text{ eV}$$

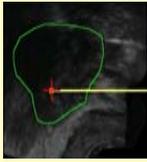
What is the maximum number of ion pairs that the ejected photoelectron is able to produce during the secondary ionization process?

$$n_{\text{max}} = 661\,000\text{ eV} / 34\text{eV}$$

$$n_{\text{max}} = 19440$$

Radiotherapy

α



Internally deposited radioactivity

β^-

Linear ion density:

e^-

the amount of ion pairs in a line generated in a unit distance (n/l)

γ

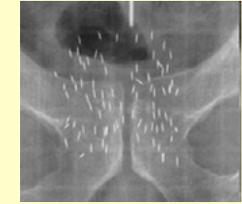
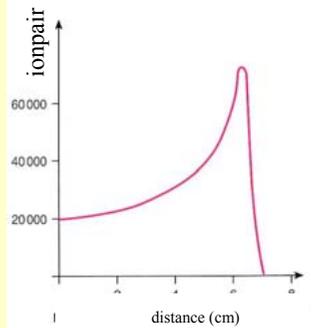
LET (Linear Energy Transfer) : the energy transferred to the material surrounding the particle track, by means of secondary electrons. ($nE_{ionpair}/l$)

Rtg,

p

n

In the air: $E_{ionpair} = 34 \text{ eV}$



α

Particle energy is not optimal

β^- :

continuous energy spectrum
typical energy: few MeV

Internally seeded radioactivity

e^- :

accelerated electron - 10-20 MeV

γ ,

production: linear accelerator

Rtg,

Efficient distance! $\approx 1 \text{ cm}/3 \text{ MeV}$

p

In the practice 6-21 Mev \Rightarrow 2-7 cm

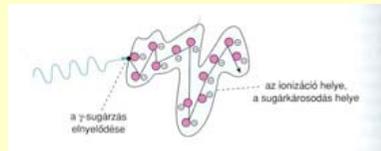
n

treatment of superficial tumours

γ : external radiation source

Site of absorption \neq sites of ionization = site of radiation damages

Penetration distance
is energy dependent



γ -knife: focused dose of radiation

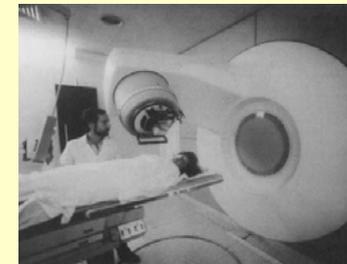
about 200 portals in a specifically
designed helmet

e.g., ^{60}Co $E_\gamma \approx \text{MeV}$,
about TBq activity



Treat tumours and lesions in the brain

X/ray:



The X-rays are generated by a linear accelerator .

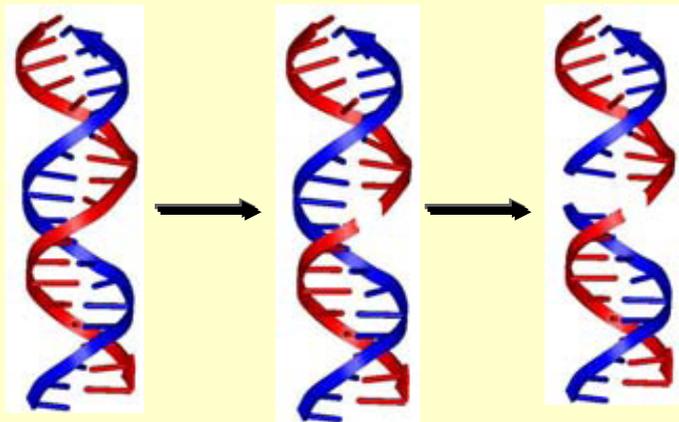
Few MeV photon energy.

2. Chemical reactions

Direct effect

Direct ionization of the macromolecules.

DNA damage is the most important!



single

double

strand breaks



chromosome aberrations

Indirect effect

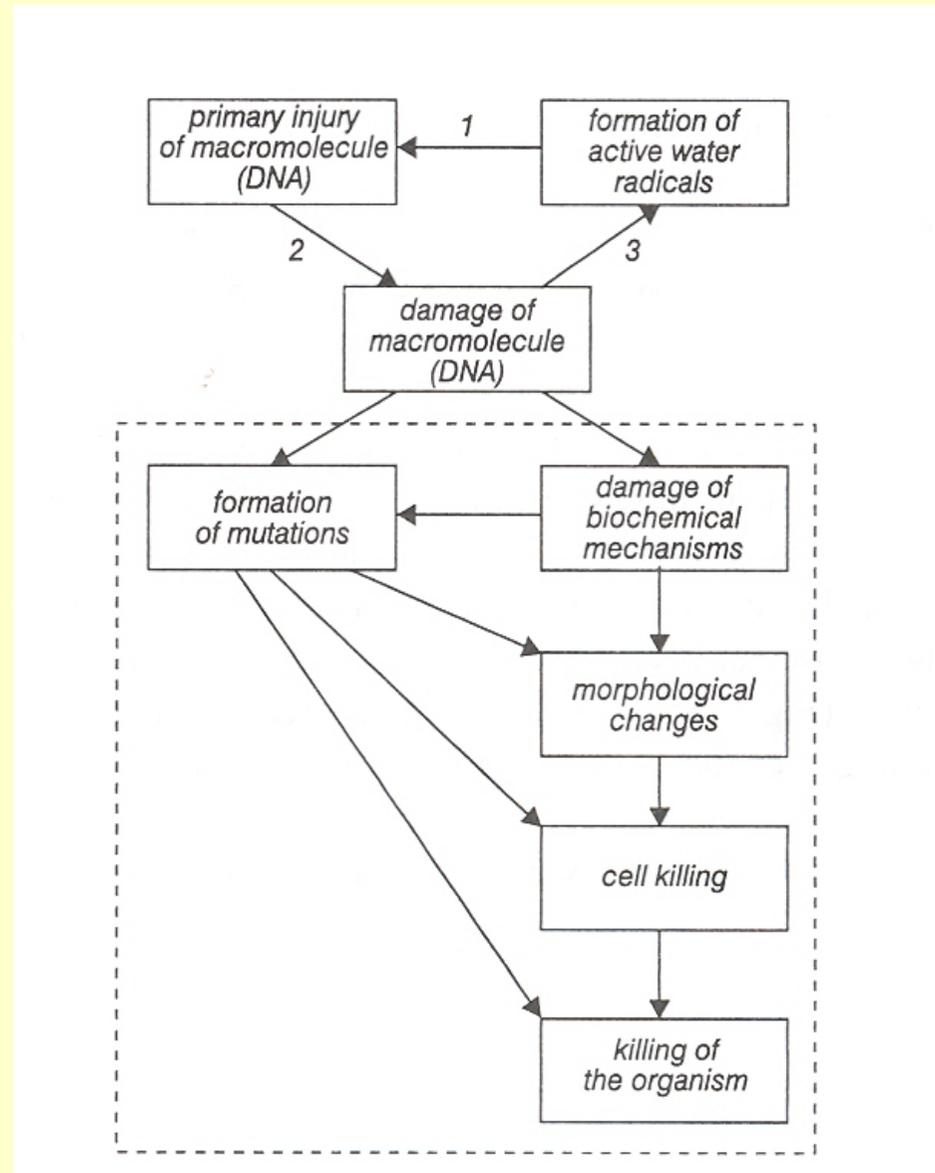
Reactive ions (e.g. OH^-) and/or radicals (e.g. $\cdot\text{OH}$) are generated mainly from water molecules.

(65-70% of the human body is water)



Reactive species induce damages in macromolecules and membrane structures.

3. Biological consequences

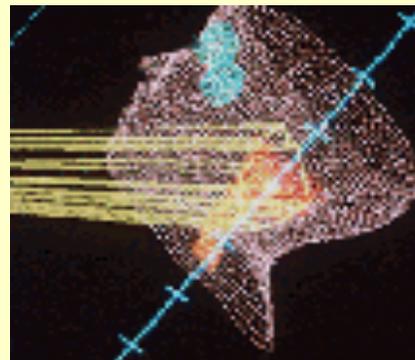


Timescale of events

Physical	$10^{-20} - 10^{-8}$ s	Ionization, excitation
Chemical	$10^{-18} - 10^{-9}$ s	Direct/indirect chemical reactions
	$10^{-3} - \text{few hours}$	Repair of damages
Early biological	hours – weeks	Cell death, death of living system
Delayed biological	years	Carcinogenesis, genetic transformation

Approaches

- **Palliative radiotherapy** to reduce pain and address acute symptoms – e.g. bone metastasis, spinal cord compression etc.,
- **Radical radiotherapy** as primary modality for cure – e.g. head and neck tumours
- **Adjuvant treatment** in conjunction with surgery – e.g. breast cancer



Ionizing radiation in radiotherapy

Electromagnetic

- X-ray – Bremsstrahlung and characteristic
- gamma
 - ^{60}Co (1,25MeV) – tele-therapy
 - ^{192}Ir , ^{125}I (35 keV), ^{137}Cs , ^{60}Co - brachytherapy

Electron/ β^- – energy range 6 – 21 MeV

Alpha - ^{225}Ac 6 MeV, ^{226}Ra 4,78 MeV

Proton – increasing use

Heavy ions – limited use

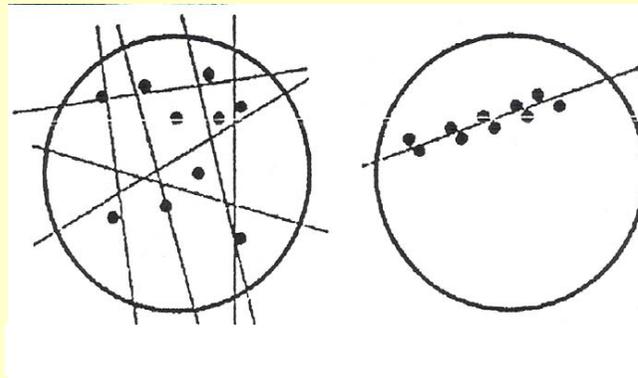
Neutron – limited use

„Efficacy” of various modalities are different

Linear ion density:

the amount of ion pairs in a line generated in a unit istance (n/l)

LET (Linear Energy Transfer : the energy transferred to the material surrounding the particle track, by means of secondary electrons. ($nE_{ionpair}/l$)



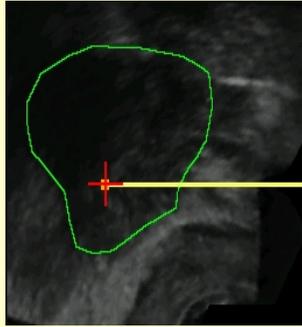
Low LET
e.g., γ , rtg

High LET
e.g., α , proton

Typical LET values

LET	Radiation	Energy(MeV):	LET(keV/μm):
high	α – particles	5.0	90
	fast neutrons	6.2	21
	protons	2.0	17
low	X-rays	0.2	2.5
	^{60}Co γ –radiation	1.25	0.3
	β – particles	2.0	0.3
	accelerated electrons	10.0	

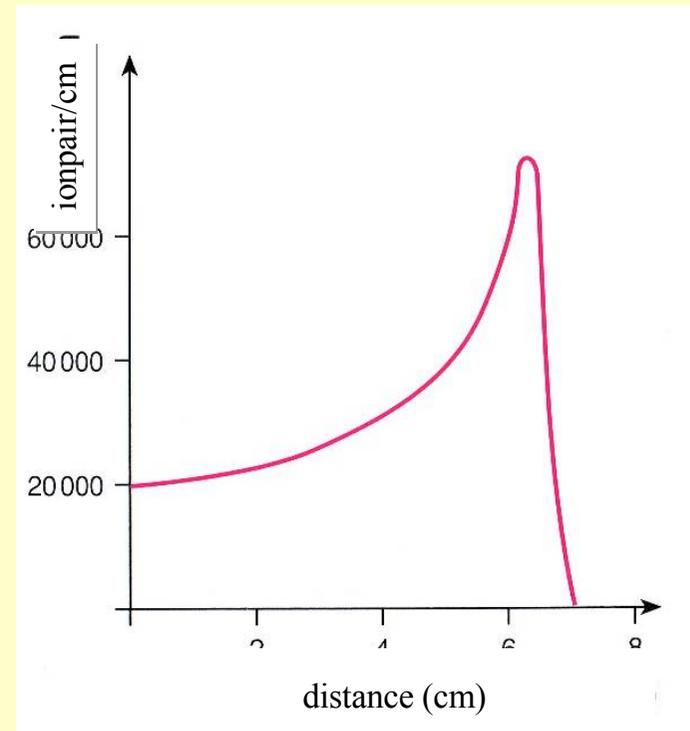
α



Internally deposited radioactivity

Brachytherapy

In the air: $E_{ionpair} = 34 \text{ eV}$



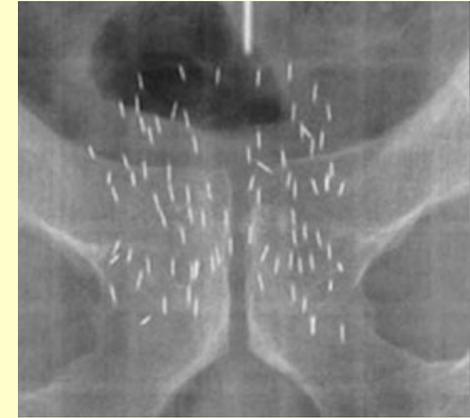
β^- :

Internally seeded radioactivity

Particle energy is not optimal

continuous energy spectrum

typical energy: few MeV



e^- :

accelerated electron - 10-20 MeV

production: linear accelerator

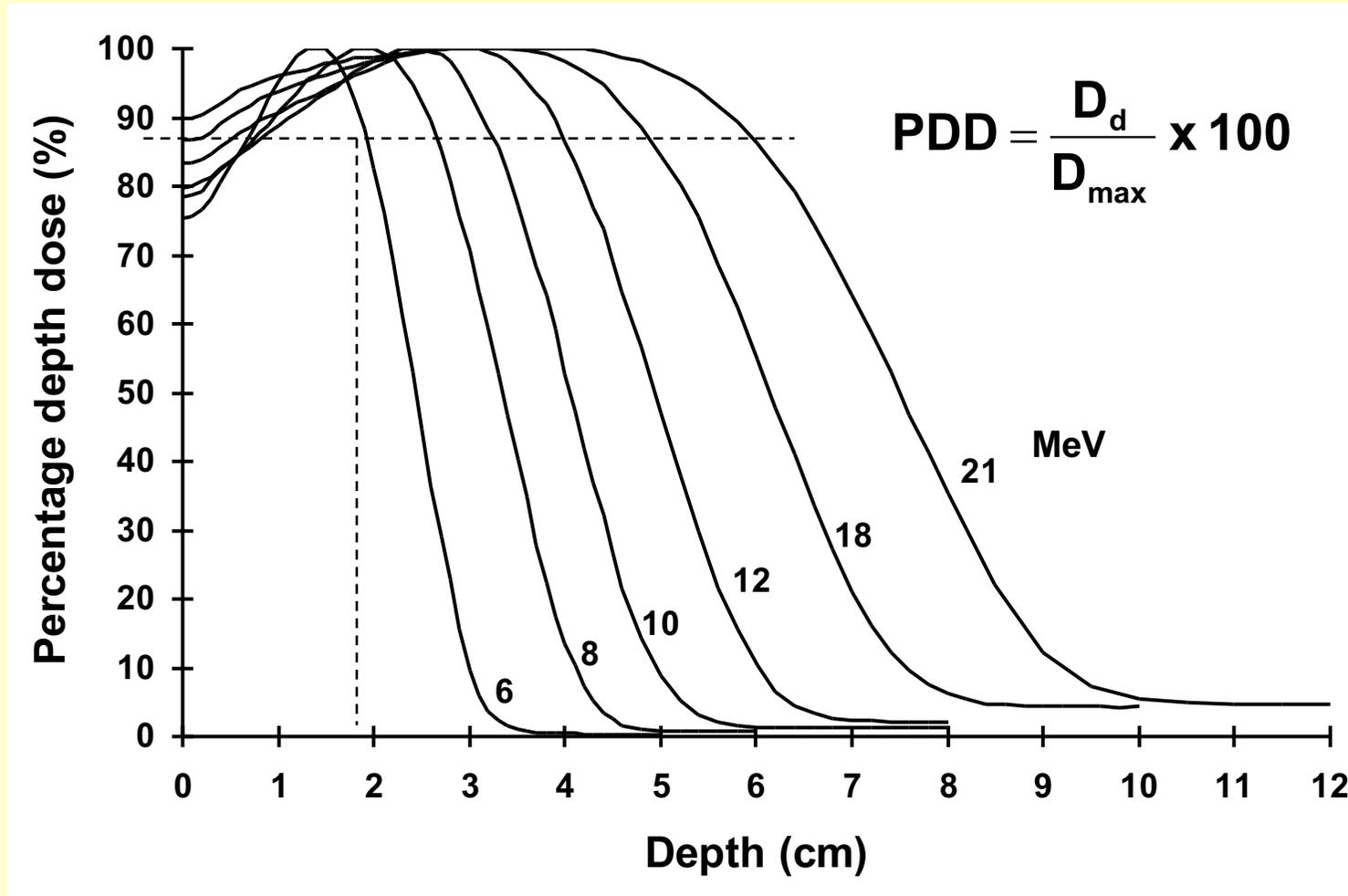
Efficient distance! $\approx 1\text{cm}/3\text{MeV}$

In the practice 6-21 MeV \Rightarrow 2-7 cm

treatment of superficial tumours

Electron PDD (percentage depth dose) curves with different energies

Reduced skin-sparing effect

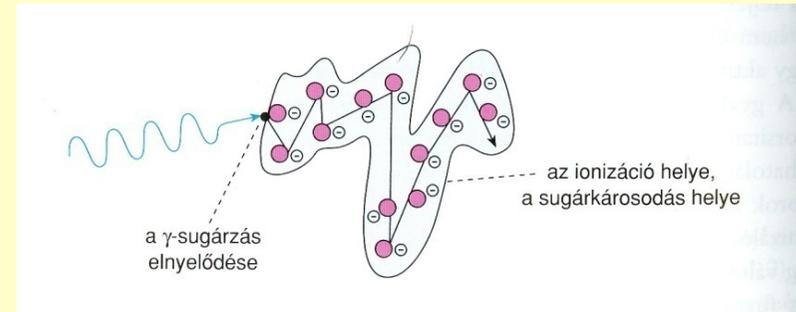


Conclusion: only superficial tumors can be treated with electron beams

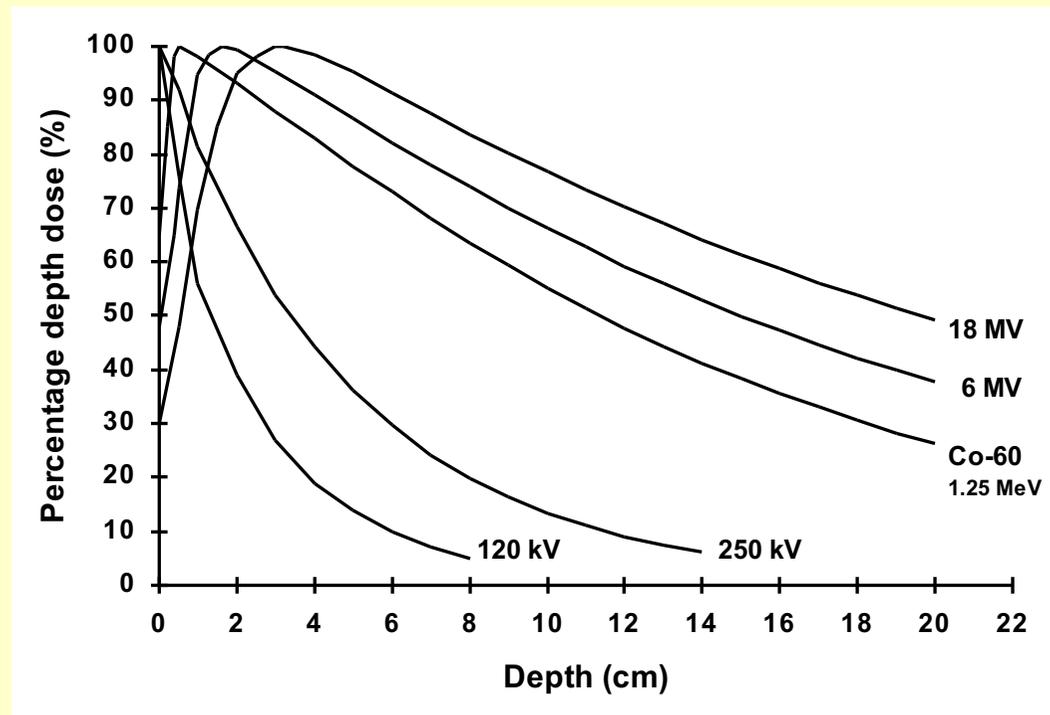
γ :

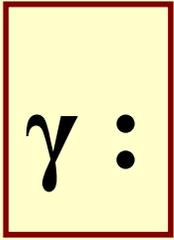
Site of absorption \neq sites of ionization = site of radiation damages

Penetration distance
is energy dependent



PDD curves at voltages
(see X-ray) and various
photon energies



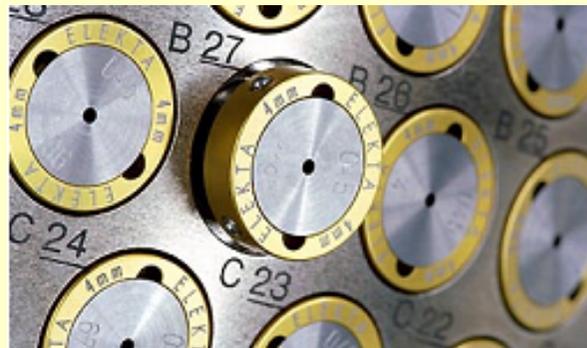
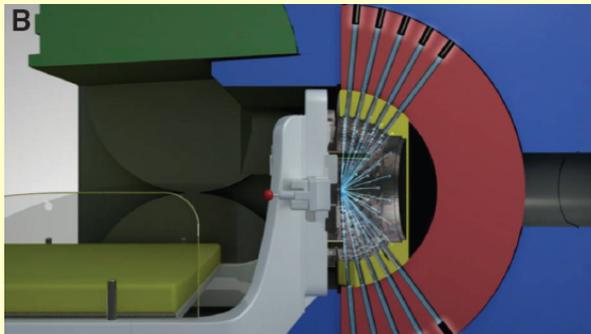


γ -knife: focused dose of radiation

about 200 portals in a specifically designed helmet

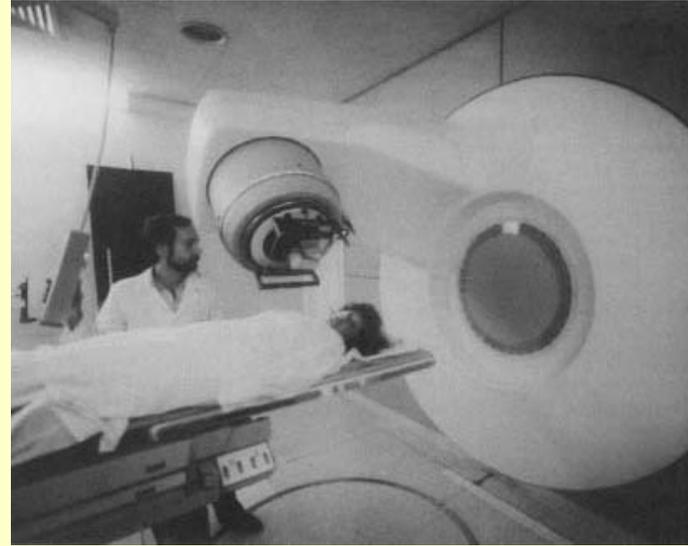
e.g., ^{60}Co $E_{\gamma} \approx \text{MeV}$, about TBq activity

The radiation isocenter is the point in space where radiation beams intersect



Treat tumours and lesions in the brain

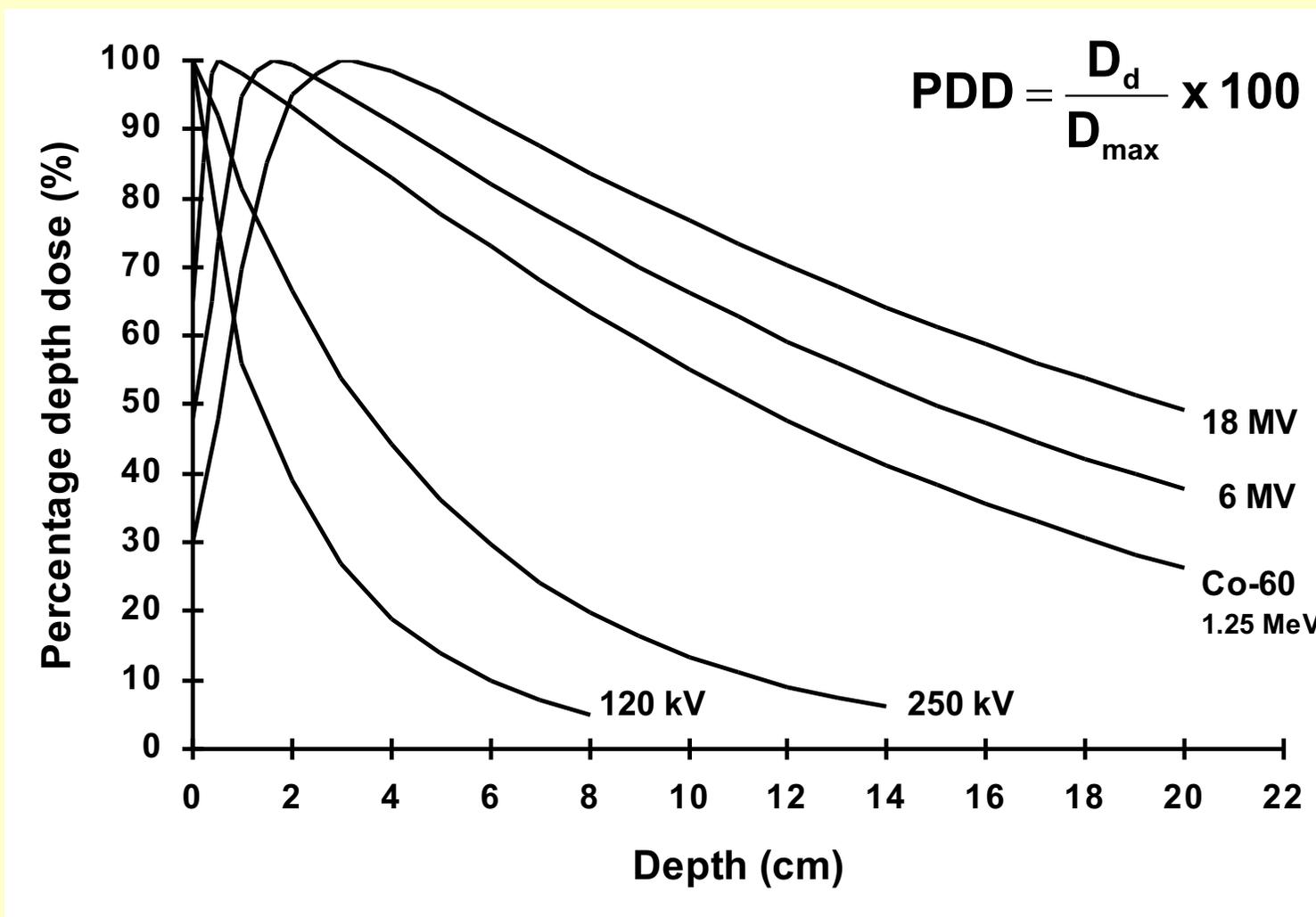
X-ray:



The X-rays are generated by a linear accelerator .

Few MeV photon energy.

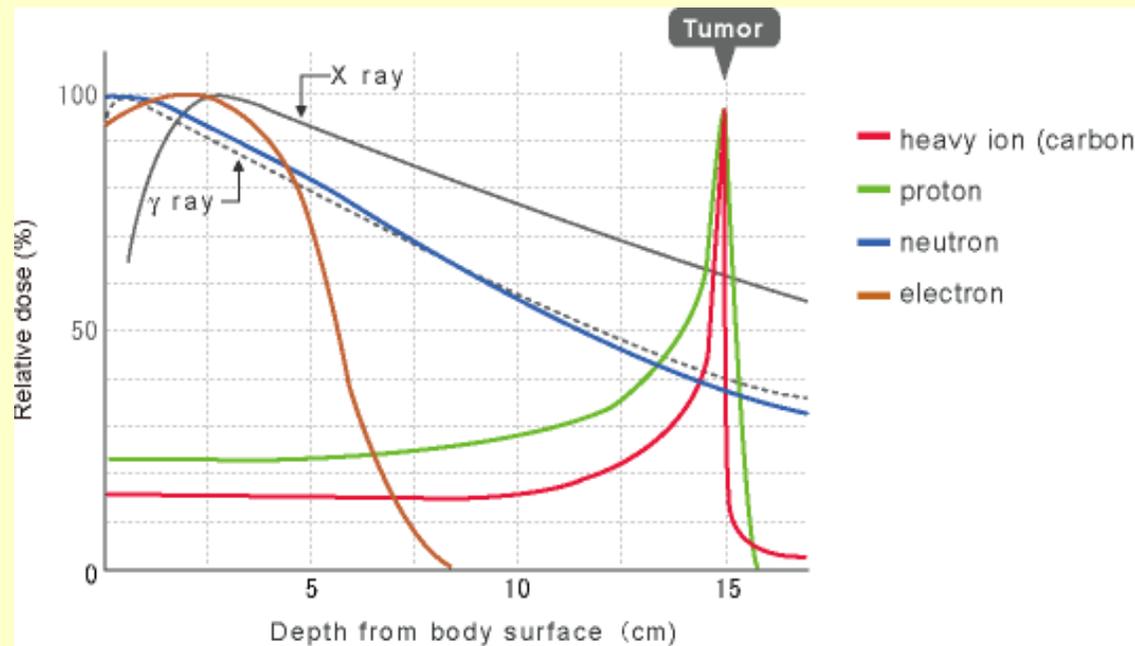
Photon PDD (percentage depth dose) curves with different energies



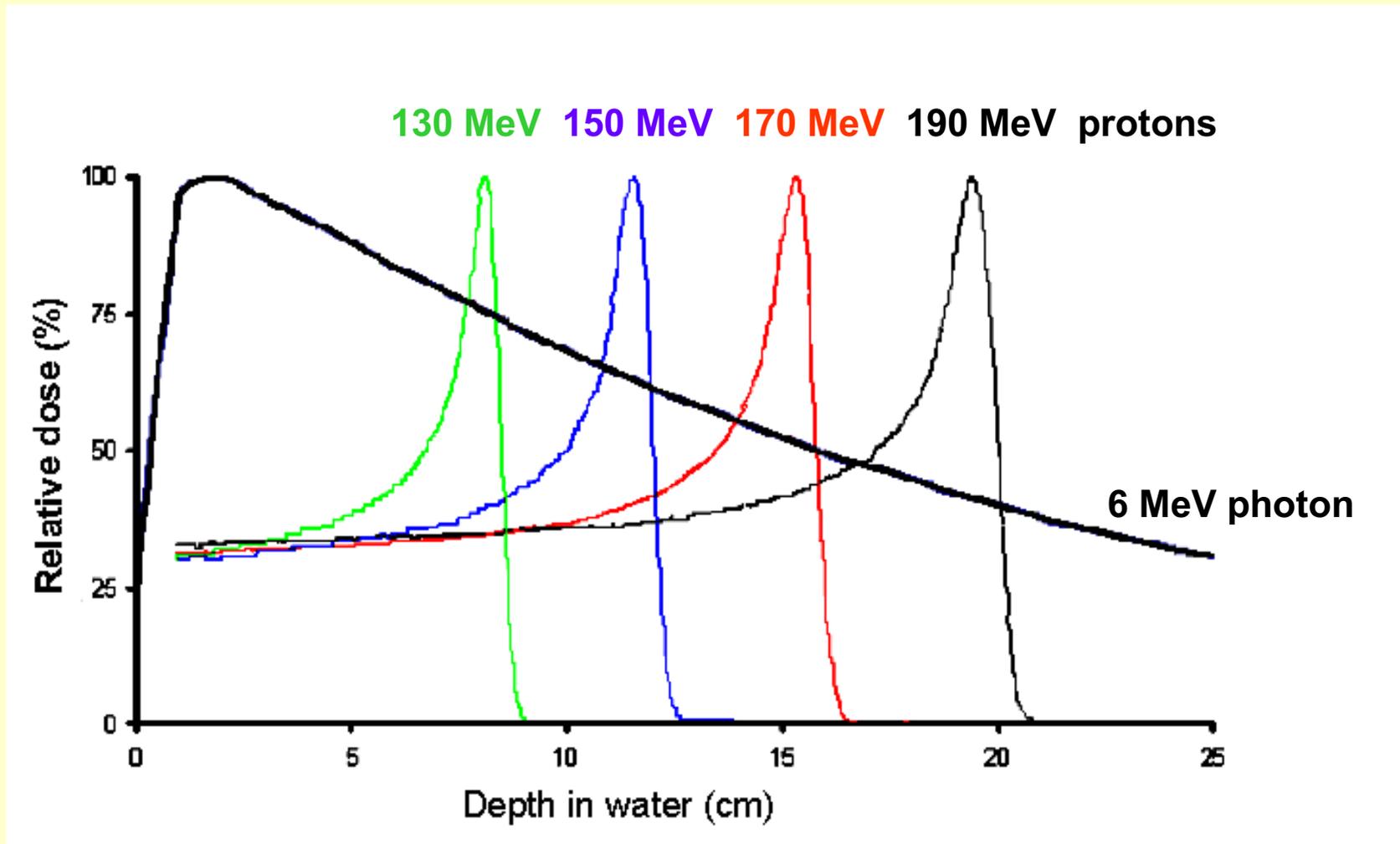


Would be ideal, but very expensive!

proton:



Comparison of photon and proton depth doses

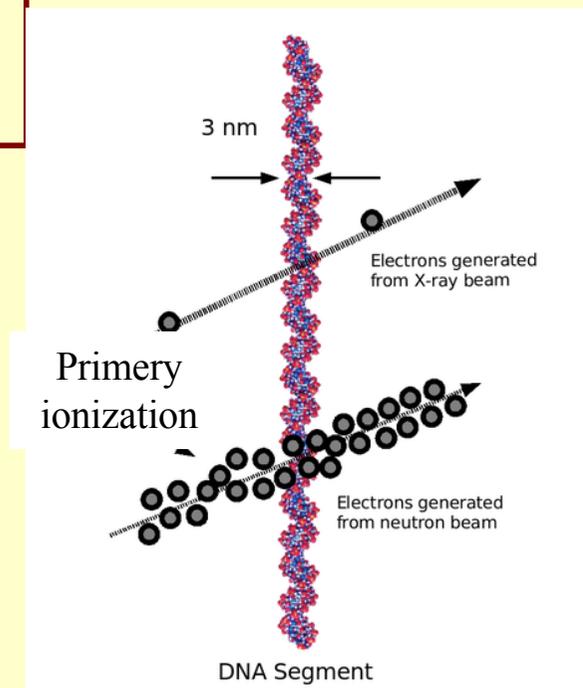
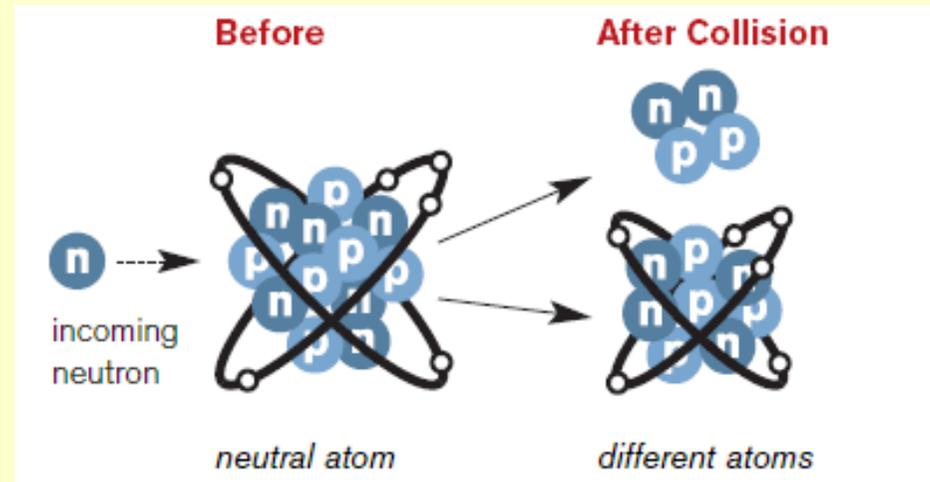


Neutron radiation: collision of high energy protons (66 MeV) into berillium target ($p(66) + Be$)

Neutrons induce nuclear reactions.

neutron:

High LET



Damjanovich, Fidy, Szöllösi „Medical Biophysics”:

II. 4.

4.1

4.2

4.3

4.4

4.5

IX.3.

Kellermayer „Medical Biophysics Practices”: Dosimetry